

Development of 90Sr-90Y generators using the cation exchange technique at IPEN/CNEN-SP

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Introduction. There is a widespread interest in the use of 90Y for various therapeutic applications, including radiolabeled tumor-specific antibodies for tumor therapy, radiolabeled particles for irradiation of malignant tumors in the liver, irradiation of solid tumors and , treatment of rheumatoid arthritis of the knee joint. 1-3 Yttrium-90 is generated through the decay of reactor-produced 90Sr (t_{1/2}=28 years) and isolated by methods like precipitation, extraction, electroclientical separation and ion exchange chromatography! Because of its simplicity, the ion exchange methods are most commonly used in the generator systems for the separation of 90Y from 90Sr invery pure form. Most of these generators have been prepared using the Dowex-50 cation exchange resins, that can retain 90Sr while the ⁹⁰Y daughter is cluted in various solvents such as citrate, oxalate. acetate, and ethylenediaminetetraacetic acid (EDTA). One must note that 90Sr is a bone-seeker and produces bone marrow depression, so the permissible 90Sr has a lifetime tolerance of 74kBq (2µCi) when fixed in bone4. The objective of this work is to develop a methodology for the preparation of 90Sr-90Y generators using a cation exchange resin method. 90Sr is strongly adsorbed in the resin and 90Y is cluted in 0.03mol.L-1 EDTA. Materials and methods. All experiments were performed at the Radiopharmacy Center at

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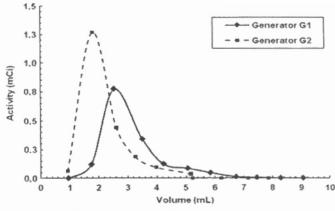


Figure 1.—Elution profile for (G $_{\rm I}$) and (G $_{\rm Z}$) $^{90}{\rm Sr}\text{-}^{90}{\rm Y}$ generators with 0.03 mol-L 1 EDTA.

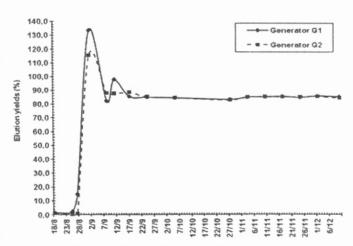


Figure 2.—Elution yields for (G1) and (G2) 90Sr-90Y generators.

IPEN/CNEN-SP, Two generators (G1 and G2) were developed, both loaded with 3 mCi (111 MBq) of 90Sr. The solution containing the pair 90Sr/90Y was prepared in 1mol.L⁻¹ HCl. The cation exchange resin used in both generators was Dowex 50W-X8 (100-200 mesh, 11* form) that was converted to Na* form with 1mol.L-1 NaO11 and distilled water. After the activation, the resin was conditioned with 100 mL of 0.003 mol.L-1 EDTA at pH= 4.55. The generator (G_1) consisted of a glass chromatographic column with a 1cm diameter and 10cm high fitted with a glass frit at the bottom. The column was assembled vertically. The flow rate was 11 drops/min. The generator (G2) differed only in the diameter, 0.8 cm with a flow rate of 6 drops/min. The elutions for both generators for separation process were performed with 0.003 mol.L-1 and 0.03 mol·L-1 EDTA. The radioactivity measurements were performed using a Capintee CRC®-15 beta dose calibrator. A liquid scintillation counting analyzer (LSC) was also available for determination of the beta emitters (90Sr and 90Y) and an HPGe detector for the determination of the gamma emitter (85Sr tracer). Pure samples of 90Y and 90Sr-90Y were used as standards in the LSC. The quality control performed for the two

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generators system consisted of following the decay of 90Y in each elution and also by measuring the activities of 85Sr (HPGe) and 90Sr (beta counting). Results, Figure 1 shows the clution profile for the (G₁) and (G₃) generators eluted with 0.03 mol.L-1 EDTA. Generator G₂ could be eluted with lower volumes (5 mL) than the generator G₁ (8 mL). Figure 2 shows the clution yields for both generators. Initially (up to 28/8), the generators were cluted with 0.003mol.L-1 EDTA with larger elution volumes and low yields. The concentration of EDTA was changed to 0.03mol.L-1, decreasing the clution volumes and increasing the clution yields. Both generators had the same behavior along the 4 months of use with average elution yields of 85%. The activity of each eluted sample was measured during 1 month for the ⁹⁰Y 1_{1/2} determination. The results showed low deviation errors of the t_{1/2} obtained in relation to the theoretical one (2.6 to 6.5%) indicating a small presence of 50Sr during the clutions. It was noticed that for generator G, the half-life started to lower in the last elutions, indicating a possible contamination with 90Sr. The same samples were analyzed for 85Sr (HPGe) and 90Sr (LSC) and it was not detected the presence of both radioisotopes, indicating a high radionuclidic purity of 90Y. Conclusion. A very promising methodology for the preparation of 90Sr-90Y generators was developed using the cation exchange technique. In this technique, 90Sr is strongly adsorbed in both generators, and 90Y eluted using 0.03 mol.L⁻¹ EDTA. The results showed that chromatographic columns with smaller diameters presented a better performance during the elutions, with smaller elution volumes and larger radioactive concentrations. Next steps include the development of a methodology to destroy EDTA, the determination of 90Sr breakthrough using an ICP-OES technique and the gradual increase of the 90Sr-90Y activity.

References

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